

## LIBRARY OF EVALUATED NEUTRON DATA FILES

A.I.Blokhin, A.V.Ignatyuk, V.N.Koshcheev, B.D.Kuz'minov,  
V.N.Manokhin, G.N.Manturov, M.N.Nikolaev

Institute of Physics and Power Engineering  
Obninsk, Kaluga Region, USSR

Abstract: The authors report on the preparation of the library of evaluated neutron data files which was recommended by the Nuclear Data Commission of the State Committee on the Utilization of Atomic Energy as a basis for improving systems of constants in neutron engineering calculations. A short description of the contents of the library and of its status is given.

A library of evaluated neutron data files has been compiled in the Institute of Physics and Power Engineering. The library contains full sets of neutron data (cross-sections of all reactions and energy and angular distributions of secondary neutrons) in the energy range from  $10^{-5}$  eV to 20 MeV. The data are presented in the ENDF/B-5 format. The library contains data for the main engineering, construction and fuel materials in reactors, radiation shielding materials, the atmosphere, soil materials and biological tissues.

Technological and construction materials:

Hydrogen isotopes	Titanium
Helium isotopes	Vanadium
Lithium isotopes	Chromium and its
Beryllium	isotopes
Boron isotopes	Manganese
Carbon	Iron and its
Nitrogen	isotopes
Oxygen	Cobalt
Fluorine	Nickel and its
Sodium	isotopes
Magnesium	Copper
Aluminium	Gallium
Silicon	Yttrium
Chlorine	Zirconium
Potassium	Niobium
Calcium	Molybdenum

Cadmium	Tungsten isotopes
Gadolinium	Rhenium isotopes
Hafnium	Gold
Europium isotopes	Lead
Tantalum	Bismuth

Actinides:

Thorium-228,232  
Protactinium-231,233  
Uranium 232-236,238  
Neptunium-237,239  
Plutonium-236,238-242  
Americium-241,242<sup>m</sup>,243  
Curium-242-248.

Fission products:

Molybdenum-95,97, 98,100	Xenon-131,135
Technetium-99	Cesium-133,135
Ruthenium-101,102, 104,106	Cerium-144
Palladium-105	Praseodymium-141
Silver-109	Neodymium-143,145
Iodine-129	Promethium-147
	Samarium-147,149, 151.

The neutron data files for the main fuel, construction and engineering materials of reactors and also for the most important nuclear fission products are compiled on the basis of recent evaluations made at the Institute of Physics and Power Engineering [1-7], at the Nuclear Energy Institute of the Byelorussian Academy of Sciences [8 and 9]

and at the Technical University Dresden (German Democratic Republic) [10 and 11]. In the case of materials, used as neutron standards, the data recommended by the IAEA were used [12]. For the remaining materials, data which Soviet specialists considered to be most reliable were taken from existing foreign files and used as a basis. In the majority of these files, the data in the resolved and unresolved resonance region were replaced by the results of the latest Soviet evaluations.

Detailed descriptions of the evaluation of most of the files contained in the library are given in the above-mentioned works. However, it should be noted that some of the files were heavily edited prior to inclusion in the library, as part of the re-evaluation carried out in 1984-1985 to take account of new experimental data. These include, primarily, the files for iron, chromium and nickel isotopes. The library's data files for natural mixtures of these nuclides were compiled from files for the individual isotopes; nevertheless they correspond to the evaluated data for the natural element cross-sections. During the review of the data for the above-mentioned construction materials, special attention was paid to re-evaluating the resonance structure of cross-sections and also the low-level excitation cross-sections for inelastic scattering. For this purpose, the opto-static description was used, the parameters of which were determined from the harmonized analysis of the total neutron cross-sections, the differential elastic scattering cross-sections and low collective level excitation cross-sections of even-even nuclei. The values obtained from this for neutron force functions at low energies do not contradict the evaluations obtained by averaging the parameters of the resolved resonances. This approach made it possible to take account of

the contributions of direct scattering mechanisms for the whole energy range and, what is important, made it possible to evaluate correctly the low-level excitation functions of odd isotopes, for which there are no direct experimental data.

The analysis of data on radiative capture cross-sections of fission products carried out during the last few years at the Institute of Physics and Power Engineering [7, 13, 14] made it possible, firstly to compile a list of the most important products determining their total capture cross-section and its evolution during burnup and decay of fragments; secondly, to select from among the available files of evaluated data (JENDL-1 and ENDF/B-5) those which correspond most closely to the present-day level of values; thirdly, to determine those nuclides for which the data needed to be reviewed to take account of new experimental information. As a result, new evaluations for the cross-sections in the unresolved resonance region were made for  $^{99}\text{Tc}$ ,  $^{101}\text{Ru}$ ,  $^{102}\text{Ru}$ ,  $^{103}\text{Rh}$ ,  $^{105}\text{Pd}$ ,  $^{107}\text{Pd}$ ,  $^{109}\text{Ag}$ ,  $^{129}\text{I}$ ,  $^{143}\text{Nd}$ ,  $^{145}\text{Nd}$ ,  $^{147}\text{Sm}$  and  $^{149}\text{Sm}$ . For isotopes where there are no direct experimental data on fast neutron capture cross-sections but only information on the mean resonance parameters, and also for isotopes where there is no information either on cross-sections or on resonance parameters, the evaluations of capture cross-sections for fission products are based on radiation force functions empirical systematics.

Fission products, for which the library has neutron data, determine more than 80% of the radiative capture by fission product mixture in all types of reactor. It is assumed that the contribution of other fission products with low capture cross-sections or low yields can be taken into account with the inclusion of group constants.

All the files were carefully checked by Soviet specialists. For the most im-

portant materials, the Nuclear Data Centre also organized a collective examination carried out by specialists from various institutes. The library includes files containing data corrected to take account of the remarks made by the experts.

Some of the shortcomings revealed have not yet been eliminated. For example, the majority of files accepted in the library need to be revised in the case of spectra for inelastically scattered neutrons and neutrons from the (n,2n) reaction taking into account the contribution of the direct and pre-equilibrium processes (at present these spectra are evaluated on the basis of the most simple evaporation model). This type of defect should not significantly affect the results of practical calculations, particularly since they appear only in reactor materials which are present in low concentrations. In future, priority will be given to revising data files with defects of this type.

The format and physical compatibility of the data in all the evaluated neutron data files in the library were checked algorithmically. The majority of files were converted into group constants by means of the GRUKON [15] and NJOY [16] programs.

Group constants were tested using the macroscopic experiments on critical assemblies [17]. The results displayed general consistency of the evaluated microscopic data.

The Institute of Physics and Power Engineering and the Nuclear Data Commission of the State Committee on the Utilization of Atomic Energy recommend the library as a basis for developing improved constant systems for neutron calculations on various engineering and scientific fields.

The work on evaluation and compilation of new files is going on for future expanded version of the library.

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